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# DIVISION

# SYSTEM DESCRIPTION

# FOR

Defueling Water Cleanup

Reactor Vessel Cleanup System

(ECA 3525-84-0041)

COG ENG EXBoldt DATE 8/27/84 RTR\_ RL Mays DATE 8/27/84

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itte	Division System Description for Defueling Water Cleanup Reactor Vessel Cleanup System	Page	2	of	23	
lev.	SUMMARY OF CHANGE					
Ē	Incorporated changes to add third 4 x 4 lon Exchanger to al exchangers to be dedicated to this system and separate syst System; revised references; revised setpoints (Section 2.2) fill and draining to reflect addition of ion exchanger leve added valve numbers.	low two em from ; revise l switc	ion FTC rd i h di	/SFP niti sabl	al e,	
2	Changed Dwg. No., item 22.b, p.7; added note p.7; deleted s "VO84 (FV-10)", p.8; deleted parentheses around "p-6 or 8", EPICOR-II from a PCBT as required," p.9; changed EPICOR to 50 ft <sup>3</sup> ENVIRALLOY Demineralizer/HIC." p.10 changed later to added note p.16.	entence p.9; a "Suclea: "4850."	p.8 Jded F Pa P.	; ad "by ckag 16;	ded ing	
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# TABLE OF CONTENTS

- 1.0 Design Description
  - 1.1 Summary
  - 1.2 References
- 1.3 Detailed System Description E.
  - 1.3.1 Description 1.3.2 System Components
  - 1.4 System Performance Characteristics
  - 1.5 System Arrangement
  - 1.6 Instrumentation and Control
    - 1.6.1 Controls
    - . 1.6.2 Power
    - 1.6.3 Monitoring
      - 1.6.4 Trips and Interlocks
  - 1.7 System Interfaces
  - 1.8 Quality Assurance

2.0 System Limitation, Setpoints and Precautions

- 2.1 Limitations
- 2.2 Setpoints
- 2.3 Precautions
- 3.0 Operations
  - 3.1 Initial Fill
  - 3.2 Startup
  - 3.3 Normal Operation
  - 3.4 Shutdown
  - 3.5 Draining
  - 3.6 Refilling
  - 3.7 Infrequent Operations
  - 3.8 Transient Operations

4.0 Casualty Events and Recovery Procedures

- 4.1 Loss of Power
- 4.2 Loss of Instrumentation/Instrument Air
- 4.3 Filter Media Rupture
- 4.4 Line and Hose Break

5.0 System Maintenance

Page 3 Rev. 2 0057Y

2-M72-DWC01

2-M72-DWC01

# 6.0 Testing

6.1 Hydrostatic Testing6.2 Leak Testing6.3 Instrument Testing

7.0 Human Factors

2

12.

#### 1.0 DESIGN DESCRIPTION

#### 1.1 Summary

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The reactor vessel cleanup system is a temporary liquid processing system which is designed to process water contained in the reactor vessel. The system's major functions are:

- a) to filter the water contained in the reactor vessel to remove suspended solids above a nominal .5 micron rating. This is done to maintain the clarity of the water to a 1 NTU rating.
- b) to remove soluble fission products from the reactor vessel contamination barrier by demineralization of the water. This is done to keep the equivalent Cs-137 concentration less than .02 µci/ml and thus reduce the dose rate contribution of the water. Also, Sb-125 concentration can be controlled by batch processing of reactor vessel water at the RCBTs.

#### 1.2 References

- Planning Study, Defueling Water Cleanup System Doc. No. TPO/TMI-046
- Technical Plan, Defueling Water Cleanup System Doc. No. TPO/TMI-047
- 3. Division I, System Design Description, Defueling Water Cleanup System Doc. No. 2-R72-DWCO1
- 4. Bechtel Drawing 2-M74-DWCO1, Defueling Water Cleanup (DWC) Reactor Vessel Cleanup System
- 5. Bechtel Drawing 2-M74-DWCO2, Defueling Water Cleanup (DWC) Fuel Transfer Canal/Spent Fuel Pool Cleanup System
- Bechtel Drawing 2-M74-DWCO3, Defueling Water Cleanup (DWC) Auxiliary Systems
- 7. Bechtel Drawing 2-POA-6401, General Arrangement Fuel Handling Building Plan EL. 347'-6"
- 8. Bechtel Drawing 2-POA-1303, General Arrangement Plenum Removal Reactor Building

- 9. DCN No. 2026-30-2, Flow Diagram Spent Fuel Cooling and Decay Heat Removal
- Burns & Roe Drawing No. 2026, Flow Diagram Spent Fuel Cooling and Decay Heat Removal
- GPU Drawing No. 2R-950-21-001 P&ID Composite Submerged Demineralizer System
- TMI-2 Recovery Division System Design Description for Submerged Demineralizer System, Doc. No. SD 3527-005
- Division System Design Description for Spent Fuel Pool/Fuel Transfer Canal Cleanup System, Doc. No. 15737-2-M72-DWC02
- 14. Instrument Index, Doc. No. 15737-2-J16-001
- 15. Design Engineering Valve List, Doc. No. 2-P16-001
- 16. Standard for Piping Line Specifications for GPU Nuclear Corp. TMI-Unit 2 Standard 15737-2-P-001
- DMI-2 Recovery Mechanical Equipment List Bechtel North American Power Corp. Job 15737
- 18. Bechtel Piping Isometrics

1

- a. 2-P60-DWC01-DWCS Pumps P-2A&B, P-3A&B, P-4A&B, and Miscellaneous Details
- b. 2-P60-DWC02-DWCS Reactor Vessel Filter Trains A & B -Inlet Manifold Piping
- c. 2-P60-DWCO3-DWCS Reactor Vessel Filter Trains A & B -Outlet Manifold Piping
- 2-P60-DWC04-DWCS Transfer Canal/Fuel Pool Filter Trains
   A & B Inlet Manifold Piping
- e. 2-F60-DWC05-DWCS Reactor Vessel Filter Train Sample Lines
- f. 2-P60-DWC06-DWCS Discharge Piping from Sample Boxes
- No. 1 & No. 2 to Penetration R-537
- g. 2-P60-DWC07-DWCS Samples Lines Upstream & Downstream of Ion Exchangers
- h. 2-P60-DWC08-DWCS Forwarding Pumps P-6 and P-7, Suction & Discharge Piping
- 2-P60-DWCO9-DWCS Forwarding Pumps P-6 and P-7 Discharge Piping
- j. 2-P60-DWC10-DWCS Supply Piping to Ion Exchangers K-1 & K-2, Supply & Discharge Piping for Post Filter F-8
- k. 2-P60-DWC11-DWCS Supply Piping to Ion Exchangers K-1 & K-2
- 1. 2-P60-DWC12-DWCS Borated Water Flush Piping from SPC-T-4

Page 6 Rev. 2 0057Y

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- m. 2-P60-DWC13-DWCS Transfer Canal/Fuel Pool Filter Trains "A" & "B" Outlet Manifold Piping
- 2-P60-DWC14-DWCS Transfer Canal/Fuel Pool Filter Trains
   "A" & "B" Outlet Manifold Discharge Piping, Supply & Discharge to Booster Pump P-5
- 2-P60-DWC15-DWCS Nitrogen Supply Piping to SPC-T-4 and Drying Station
- p. 2-P60-DWC16-DWCS Discharge Piping from DWC Booster Pump P-5
- q. 2-P60-DWC17 DWC Miscellaneous Piping Details
- ECA No. 3525-84-0041, Definition of the Defueling Water Clamaup System.
- ECA No. 3245-84-0034, Defueling Water Cleanup System Penetration Modifications.
- 21. ECA No. 3527-84-0042, SDS Tie-in to DWCS.
- 22. Bechtel Area Piping Drawings
  - a. 2-P70-DWCO3 DWCS Hose Network Reactor Bldg. Plan El. 347'-6"
  - b. 2-P70-DWC04 DWCS Hose Network Fuel Handling Bldg. Plan E1. 347'-6"

#### 1.3 Detailed System Description

#### 1.3.1 Description

The reactor vessel cleanup system is a liquid processing system which will process water from the reactor vessel. The system is shown schematically on Dwg. 2-M74-DWCO1, and its associated Dwgs. 2-M74-DWCO2 and 2-M74-DWCO3. (Note, some valves identified herein have been given an instrument designator as well as a valve number. When this occurs, the instrument designator is shown in parentheses after the valve number.)

The system has two submersible type pumps (deep well pumps), P-2A and 2B, which are housed in wells and located in the fuel storage pit in the shallow end of the fuel transfer canal in the Reactor Building. Each pump has a 220 gpm capacity and will process 200 gpm from the reactor vessel and recirculate 20 gpm. The suction from the reactor vessel is through the Westinghouse work platform via hoses which connect the nozzles provided on the work platform to the wells.

> Page 7 Rev. 2 0057Y

The system has four particulate filters, F-1, 2, 3 and 4, each capable of filtering a flow of 100 gpm. The filters are composed of sintered metal filter media which is contained in modified fuel canisters. These filters are capable of removing debris, mainly fuel fines  $(UO_2)$  and core debris  $(2TO_2)$ , down to a 0.5 micron rating. Since the canisters contain fuel fines, they are designed to prevent a criticality condition from existing when they have been loaded. Also, the filters are submersed in the transfer canal to provide the appropriate radiation shielding.

The two pumps and four filters are arranged so that one pump discharges to two filters. Therefore, the filtration portion of the system is divided into two trains, each train contains one pump which feeds two filter canisters. This allows the system to filter 200 or 400 gpm from the Reactor Vessel. Normally, the system will process 400 gpm of Reactor Vessel water. The two pump arrangement allows for greater flexibility in system operations and provides redundancy to allow system operation during maintenance.

A filter is used continuously until the differential pressure reaches a predetermined setpoint. At this point the system is shutdown and then, after a waiting period (approximately 5 min.), it is restarted. The differential pressure is noted and if it returns to a low value the system will be run again to the pressure setpoint. This process is repeated until the differential pressure at restart reaches a value near the shutdown setpoint. When this occurs within one hour, the train is shutdown and the filters are replaced.

Once the water has been filtered, all, or a portion of, the flow can be returned to the Reactor Vessel. The amount of water returned is controlled by remotely adjusted valves VGISA  $\delta$  B (HV30A $\delta$ B). Each of these lines will connect, via flexible hoses, to the separate inlet nozzles on the work platform. A sparger has been placed on each return line to maintain a positive pressure in the attached hoses.

That portion of the flow not returned to the vessel can be further processed to remove soluble fission products. The flow from the pumps, which passes through a single line to the ion exchangers, is automatically controlled by remotely adjusted flow control valve V084 (FV-10). There is a post filter, F-5, located in that line to ensure that, in case of a failure of a filter canister, no fuel fines can reach the ion exchangers. The loading on the post filter is expected to be minimal and, as such, they will be standard disposable cartridge type filters.

> Page 3 Rev. 2 00575

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Two ion exchangers, K-1 and K-3, are dedicated for use in this system. Each of these ion exchangers can handle the normal 30 gpm flow. The ion exchange media is a bed of zeolite resin which will remove the Cs-137. The two ion exchangers provide flexibility in operation so that one can be taken out of service without interrupting normal flow. The ion exchange media is contained in a 4 x 4 liner which is similar to those in use for the EPICOR II system.

The level in each ion exchanger is maintained by regulating the flow out of the ion exchanger to match the inflow. The outflow is regulated by a throttling control valve V029 or V266 (LV 45 or 72), which is located downstream of an air driven reciprocating diaphragm pump P-6 or 8. This valve automatically throttles in response to changing level in the ion exchanger. The maximum system flow rate is 30 gpm per 4 x 4 liner. The pumps head-flow characteristics can be changed by adjusting the air pressure to the pumps. Post filter F-6 is located downstream of these pumps to prevent the migration of any resin fines. The water is returned from here to the reactor vessel via flexible hoses.

Sampling points are provided at various locations in the system. The samples are routed to two sample glove boxes which are located in the Fuel Handling Building. This system provides the operator with the capability to periodically monitor the effectiveness of the system. Also, the turbidity of the effluent from the filters is constantly monitored by nephelometers and displayed at the local control panel. The radiation levels of the ion exchange influent and the boron concentration and pH of the ion exchange effluent are also constantly monitored and displayed at the local control panel.

Several inlets have been provided on the DWC system through which borated water can be gravity fed from the standby reactor pressure control system storage tank to backflush the system. The system will be backflushed when radiation levels in the piping are determined to be excessive and prior to maintainence.

A path to allow flow to the reactor coolant bleed tanks is provided to allow for system inventory reduction. Also, batch processing to remove Sb-125 will be performed by EPICOR-II from a RCBT as required. This flow path uses a portion of the submerged demineralizer system. This path is located downstream of the DWC ion exchangers, and, as such, this flow does not pass through the SDS ion exchange vessels.

> Page 9 Rev. 2 0057Y

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# 1.3.2 System Components

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#### P-2 A/B Reactor Vessel Cleanup Pumps

Type: Vertical Submersible Deep Well Pump Model: Goulds VIS 9AHC/2 Material: Stainless Steel Bowl and shaft with a bronze impeller Motor: Franklin Electric 25 hp, 460 Volt, 3 phase Rating: 264 FT TDH at 220 gpm Minimum Flow: 20 gpm

# F-1/2/3/4 Reactor Vessel Filters

Type: Pleated Sintered metal media Model: Pall Trinity special product contained in a critically safe canister Rating: 0.5 micron Nominal Removal Rating Flow: 100 gpm

#### F-5/6 Filter Canister Post Filter and DWC Post Filter

Type: Disposable Cartridge Model: Filterite No. 921273 Type 18M503C-304-2-FADB-C150 Rating: 0.45 micron nominal removal rating Flow: 20 to 60 gpm

#### K-1/3 Ion Exchangers

Type: Zeolite resin contained in a 4'x4' HIC Model: Nuclear Packaging 50 ft<sup>3</sup> Enviralloy Demineralizer/HIC Flow: 30 gpm

#### P-6/8 Forwarding Pumps

Type: Air driven double diaphragm pump Model: B.A. Bromley Heavy Metal Pump Model No. H25 Material: Stainless Steel with Viton diaphragms Rating: 60 feet TDH at 60 gpm

For instrumentation, valves, piping. and equipment details, see references 14, 15, 16 and 17, respectively.

#### 1.4 System Performance Characteristics

The system is designed to function in any of the modes of operation shown in table 1 below.

Page 10 Rev. 2 0057Y

 FILTER FLOW (GPM)
 ION EXCHANGER FLOW (GPM)

 [Return to
 [Return to

 Reactor Vessel;
 Reactor Vessel]

 400 [200]
 0

 370 [170]
 30

 340 [140]
 60

(Numbers in brackets indicate flow if only one train is in operation.)

#### Table 1

Reactor Vessel Cleanup System Operational Configurations

The operational mode is determined based upon the solids loading in the reactor vessel. Normally, 400 gpm from the reactor vessel is filtered, and 30 to 60 gpm from the reactor vessel is demineralized. The 30 to 60 gpm is chosen based on the equivalent Cs-137 concentration in the reactor vessel.

As the filters load up, the pressure differential across the filter train increases. As the differential pressure increases, the flow rate is maintained constant by manually adjusting remote valves V015A and V015B (HV-30A and 30B).

#### 1.5 System Arrangement

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Well pumps P-2A and 2B are located in the fuel storage pit of the Unit 2 Reactor Bldg. These pumps are housed in wells which are located in this pit. The wells are connected by hose to the Westinghouse work platform. The pump discharge is routed to the filter canisters via a skid mounted manifold which is located above the water level of the canal. The filter isolation valves are also located on the gkid. The filter canisters are in racks which are submersed in the fuel transfer canal. The manifold is connected to the inlet and outlet of the filters via coded, armoured hose. The inlet and outlet connections are coded to prevent mis-connection of the hoses. The outlets from the filters return to the manifold from where the water is routed back to the vessel or to the ion exchange system for further processing.

> Page 11 Rev. 2 0057Y

The ion exchangers are located behind appropriate shielding in the Fuel Handling Building. The water that has been demineralized is pumped back to its source by air driven pumps which are located near the ion exchangers.

The system uses the following existing penetrations which have been modified for their temporary function.

Pupetration No.	Suctom	Modified
renetiation No.	578100	Tuletton
R-542	Decay Heat	Backflush
R-546	Radwaste Disposal Gas	Flow to Ion Exchangers
R~553	Radwaste Disposal Reactor Coolant Liquid	Return from Ion Exchangers
		Modified
Penetration No.	System	Function
R-537	R.B. Dergency Spray and Core Flooding	Sampling Return
R-54586C	Spares	Sampling

For further location and arrangement information, see references 7, 8 and 18.

# 1.6 Instrumentation and Control

# 1.6.1 Controls

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The majority of system control is handled remotely from a control panel which is located in the Fuel Handling Building. This is due to the fact that much of the system is located in the Reactor Building which has limited access. The reactor vessel cleanup pumps do have local hand switches to shut the pumps down.

Filtered water flow back to the reactor vessel is monitored by the operator and adjusted by remotely controlled valves VO15A and VO153 (HV30A6B). The flow to the ion exchangers is controlled automatically by flow control valve VO84 (FV-10), which seeks to maintain flow to the ion exchangers at the selected setpoint.

> Page 12 Rev. 2 0057Y

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R-553	Radwaste Disposal Reactor Coolant Liquid	Return from Ion Exchanger
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R-54586C	Spares	Sampling

For further location and arrangement information, see references 7, 8 and 18.

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> Page 12 Rev. 2 0057Y

Return flow to the reactor vessel from the ion exchangers is controlled automatically by control valves V029 and V266 (LV 45 and 72). Each valve responds to a changing level in its associated ion exchanger, which is sensed by a "bubbler" type of level measuring device. This level is maintained between a high and low setpoint in the ion exchanger. The forwarding pumps' head-flow characteristics can be changed as needed by manually adjusting the air pressure to the pumps. This control scheme maintains the inflow equal to the outflow in the ion exchanger.

For further information on the instrumentation, refer to the Instrument Index (Ref. 14).

1.6.2 Power

The pump motors are supplied with 480V power through a motor control center which is energized by an existing unit substation located in the Auxiliary Building. 120 VAC power will be supplied from the control panel or local sources.

# 1.6.3 Monitoring

Monitoring equipment is provided to evaluate the performance of the system and to aid in proper operation of the system.

The discharge pressure of the submersible well pumps is monitored (PI-4A & 4B) to determine if the pump is operating correctly and also to provide another indication that the pump is operating.

In order to determine the degree of filter loading, the primary filter canisters and the secondary post filters are equipped with remote indication of differential pressure across the filters (DPI-5A & 5B). The differential pressure across the canisters will be used to determine when the filters are loaded to capacity.

Flow in the filter train and the ion exchanger loop is monitored (FI-7A, 7B and 10) to ensure that inflow to the reactor vessel equals the outflow. This is important to insure that the level in the reactor vessel remains constant. Also, the flow to each ion exchanger is integrated by a local device (FQI76 & 77) to determine the loading on each ion exchanger.

> Page 13 Rev. 2 0057Y

The process fluid conditions are monitored to determine the effectiveness of the system. The turbidity level in the fluid is monitored (Al 43A & 43B) prior to its return to the source. Also, the capability to obtain grab samples of process fluid has been provided for at several locations in the system. The radiation levels in the process fluid are continuously monitored upstream of the ion exchargers (RE-9). A high radiation alarm (RAH-9) is also provided. This will alert the operator to the need of processing at a higher rate. The pH and boron concentration are monitored downstream of the ion exchangers (AE 11 & 12). This assures the operator that the proper boron concentration is maintained in the return flow.

The high and low level trip conditions in the ion exchangers alarm to the local control panel located in the Fuel Handling Building. This is done to ensure that the operator is aware of this condition so that the proper evaluation of the system condition that caused the level mismatch can be made. The operator will disolute the ion exchanger portion of the system during this event.

# 1.6.4 Trips and Interlocks

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The reactor vessel cleanup well pumps, P-2A/B, are provided with low level setpoint trips to ensure that the pumps do not operate under potential cavitation conditions. Also, a low level in an ion exchanger will trip its forwarding pump and terminate flow to that ion exchanger. Additionally, a high level in the ion exchanger would terminate flow to that ion exchanger by closing its inlet valve, and also would stop its forwarding pump. These trips are taken from a conductivity level probe in the ion exchangers. A low level in the IIF will trip pumps P-2A and P-2B.

Locally mounted toggle switches are provided near the ion exchanger to disable the level switches to allow for filling and draining of the ion exchangers. An alarm at the local control panel alerts the operator when the level switches are disabled.

The reactor vessel cleanup well pumps, P-2A/B are equipped with interlocks to prevent them from being started during a low level condition. Additionally, either a high or s low level in the ion exchangers prevents the valve upstream of the ion exchangers from being opened and

> Page 14 Rev. 2 0057¥

prevents the restart of the air driven forwarding pumps. This will ensure that the exchangers are not overfilled and also that any flow mismatch condition is properly evaluated by the operator prior to restart of the system.

For trip setpoints, see section 2.2.

#### 1.7 System Interfaces

Those systems interfacing with the DWC are as follows:

- a) Standby Reactor Pressure Control System (existing)
   Use: Provide a source of borated water for backflushing
   Tie-in: A single connection from SPC-T-4 downstream of SPC-VI
   to several points in system
- b) Submerged Demineralizer System (existing) Use: Provide a path to the reactor coolant bleed tanks Tie-in: Downstream of CN-V-IX-32 near 1 1/2" jumper from downstream of ion exchanger post filters.
- c) Instrument Air System (existing)
   Use: Provide source of instrument air to equipment in the Reactor Building.
   Tie-in: From existing Instrument Air supply to LOCA dampers.
- d) Instrument Air System (existing) Use: Provide source of instrument air to equipment located in the Fuel Handling Building. Tie-in: From existing Instrument Air supply to Spent Fuel Pool gate seals
- e) Service Air System (existing)
   Use: Provide a source of service air to the Forwarding Pumps P-6 and P-8.
   Tie-in: Service Station 87 plus another station if needed

#### 1.8 QUALITY ASSURANCE

The defueling water cleanup system is classified according to the safety functions of its parts. There are three classifications in this system:

a. Portions of the system associated with ion exchange processing are considered to be a radioactive waste processing system; therefore, these portions of the system shall be subject to the quality assurance guidelines contained in NRC Regulatory Guide 1.143.

> Page 15 Rev. 2 0057Y

- b. The filter canisters are classified as nuclear safety related and are designed to prevent a condition that could result in a return to nuclear criticality of the fuel retained in the filters.
- c. The remaining portions of the system are subject to the BNAPC uon-safety-related quality assurance program.

The TMI-2 Recovery QA Plan will be applicable for work performed on site.

#### 2.0 SYSTEM LIMITATIONS, SETPOINTS AND PRECAUTIONS

#### 2.1 Limitations

The system is flow limited to 200 gpm through each filter train, 400 gpm total, and 30 gpm through each ion exchanger, 60 gpm through the ion exchanger loop.

The main filter canisters are limited to 45 pai pressure differential. At this point an alarm on the local control panel will inform the operator to stop and restart the system or change out the filter.

The post filters are limited to 10 psi pressure differential. At this point, the filters are considered fully loaded and are changed out.

# 2.2 Setpoints

DPSH 5 A/B trips the alarm at 45 paid across the filter canisters.

ASL 12 trips the alarm at a low Bcron concentration of 4850 ppm.

DPSH 33 trips the alarm at 10 paid acreas the filter canister post filter.

RC-LIS 103 trips alarms and pumps at high IIF level of 327'-9" and a low IIF level of 327'-3".

RIS-9 trips the alarm at a high radiation reading of 0.9 mR/hr.

LSL 37/71 and LSH 37/71 trip the ion exchange loops at  $\pm 3^{\circ}$  from the normal operating level. (Note: Low level is  $10^{\circ}$  from top of IX, High level is 4° from top of IX.)

LIS 2A/B trip pumps 2 A/B at a decreasing level of  $1^{-0}$  above the auction of the pump.

For additional setpoint information, refer to Ref. 14.

Page 16 Rev. 2 0057Y 2

#### 2.3 Precautions

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Due to the number of quick disconnect couplings, extra care should be taken to ensure that the couplings are properly connected and that they are connected in the proper locations. This precaution will help prevent a loss of reactor vessel inventory.

The portion of the startup procedures concerning the well pumps should be strictly adhered to to prevent the rapid filling of an empty manifold. This situation could cause a harmful pressure wave to develop which has the potential to damage the filter media.

The filter canisters operate by a surface filtration method, and their efficiency increases as a cake is built up on the surface of the media. Therefore, the build up of this cake is an important part of the filtration process. To prevent the migration of fines to the post filter, the ion exchange portion of the system should not be started until a cake has begun to be formed on the media. This can be verified by observing the turbidity of the filter effluent. When the filter train is started up, there will be an initial turbidity spike caused by smaller particles passing through the media. As the cake is built, these particles are stopped and the turbidity decreases. Once the turbidity reaches a level of 10 NTU or less, the ion exchange portion of the system can be started. To prevent the breakdown of the cake, the system should not be started or stopped unnecessarily.

Due to the radioactivity in the water, the system should be carefully drained and flushed prior to any maintenance work.

Double isolation has been provided for in the system design to separate borated and nonborated water supplies.

#### 3.0 OPERATIONS

#### 3.1 Initial Fill

The system is filled initially by borated water from the standby reactor coolant pressure system through the backflushing system provided (see section 3.7). The filters are filled to the inlet and outlet manifolds and the ion exchangers are filled until they reach their normal operating level.

To initially fill the DWC ion exchangers K-1 and K-3 the level switches must be blocked out. This is accomplished by placing the block-out toggle switch, for the ion exchanger to be filled, in the proper position. This will allow isolation valve V025 or V260 to be opened. Once the ion exchanger is filled to the operating level,

> Page 17 Rev. 2 0057Y

the level switches are returned to operation by returning the block out toggle switch to its original position. As a reminder to the operator, an alarm is activated at the control panel when the level switches are blocked out.

## 3.2 Startup

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Prior to startup, valve alignment must be checked to verify that the process water for each filter train and the ion exchanger loop is taken from and returned to the contamination barrier. The well pumps are isolated by the remote manual isolation valves. The ion exchanger loop is isolated by the remote isolation valves and the control valve in the ion exchanger loop. The return lines to the source are isolated by the remote control valve. The well pumps are started and placed on minimum recirculation flow. The pump isolation valve for one filter train is slowly opened to allow any trapped air to escape through the automatic vent valves. Once the isolation valve is fully opened, flow is started by slowly opening the return line to the source. Flow through this train is allowed to reach the desired flow and then the other train is brought to the desired flow by following the same procedure.

The ion exchanger is brought into service only after the filtration system is in operation. The ion exchange system is brought into service only after the initial turbidity spike, as monitored by the in line nephelometer, reaches an acceptably low level (i.e. 10 NTU). The ion exchanger should always be brought to normal operating level prior to operation of this portion of the system.

If the ion exchanger is at low level, it should be filled with borated water from the back flush system (see Section 3.1). At this time, the inlet isolation valve is opened. The air pressure to the ion exchanger forwarding pump is manually adjusted to the pressure required to maintain the flow rate chosen. The air supply solenoid valve is then opened which will start the pump and lower the level in the ion exchanger to the point where the level control valve will close. This level is above the isolation low level. Flow is slowly started to the ion exchanger flow rate is obtained. The proper flow of water will be returned to the source automatically as the level control for the ion exchanger adjusts the downstream control valve to regulate the outflow.

#### 3.3 Normal Operation

Normal operation of the system is in one of the modes shown in Table 1 of Section 1.4. The mode of operation chosen is based on the particulate and radioactivity concentrations in the Reactor Vessel.

> Page 18 Rev. 2 0057Y

# 3.4 Shutdown

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The steps to bring the system to a shutdown condition are basically the reverse of the startup procedure. The ion exchanger flow would be brought to zero slowly by remote operation of the upstream flow control valve. As the level in the ion exchanger drops, flow from the forwarding pumps is terminated by the closure of the control valve downstream of the pump. Either (depending on system operation) of the isolation valves from the filtration trains to the ion exchanger is then closed, and the solenoid valve for the air supply to the forwarding pump is closed. The filtration trains are shut down one at a time by closing the flow return control valve and then shutting down the pumps and closing the pump isolation valves.

#### 3.5 Draining

There is a low point drain which can drain both filtration train manifolds and part of the ion exhcange loops to the fuel transfer canal. A manual blockout switch is provided which will deactivate the ion exchanger level switches. This allows the ion exchangers to be pumped out to the reactor coolant bleed tanks via the connection provided.

## 3.6 Refilling

The fully drained system can be refilled in the same manner that the system was initially filled. A partially drained system can be refilled by using either the back flush system (see section 3.7) or the well pumps (see section 3.2).

# 3.7 Infrequent Operations

Flushing of the system may be performed when the internal contamination level gets high or prior to internal maintenance work. The system is shutdown (see Section 3.4) prior to flushing.

One flushing option allows a gravity flush from SPC-T-4. Borated water is stored in the charging water storage tank, SPC-T-4, located at the 347 ft. elevation in the Fuel Handling Building. This tank is connected to the DWCS. Either filter train may be flushed without stopping flow through the other.

Flushing may be accomplished by opening one of the inlet values from the flushing system (depending on which portion of the system is to be flushed) and then opening the drain value to the fuel transfer canal. After sufficient time has been allowed to flush the system, the drain value is closed and then the inlet value is closed. The system is then restarted following the procedures in Section 3.2.

System inventory can be decreased as needed by diverting the return flow from the ion exchangers through the lines provided to the Reactor Coolant Bleed Tanks. Also, water can be routed to the RCBT as required for processing to remove Sb-125.

> Page 19 Rev. 2 0057Y

# 3.8 Transient Operations

The only effects of anticipated transients (e.g., loas of pumps, valve misoperation, or loss of controls) are a mismatch of flows from and to the reactor vessel. More water can be removed from the source than is replaced by a failure to control level in the ion exchangers. In the case where level control in the ion exchangers is lost, the source outflow will exceed the inflow by a maximum of 60 gpm. This situation can be detected by flow instrumentation in the return line and by a decreasing level in the reactor vessel. This is an unlikely event since the level controls receive their signals from two separate, diverse types of level instrumentation.

#### 4.0 CASUALTY EVENTS AND RECOVERY PROCEDURES

#### 4.1 Loss of Power

A loss of power to the entire system would simply shut the system down. A loss of power to the well pumps with an additional failure which results in simultaneous loss of level control in the ion exchangers would result in a flow mismatch. In this case, the system would be automatically shut down until power is restored. Loss of power to individual components would place the component in its safe mode. An air operated valve, for example, would fail to a position that ensures no damage to other components.

Loss of power to the control panel would cause the loss of all information and fail all control and solenoid operated valves. The system would be shutdown until power is restored.

#### 4.2 Loss of Instrumentation/Instrument Air

Loss of instrumentation would hamper operations but no adverse conditions would result and the system could be safely shut down until the problem is resolved.

Loss of a aingle instrument channel will result in the loss of indication for that channel and, for those channels that have control features a flow mismatch. This flow mismatch will result in an automatic shutdown of the affected portion of the system.

Loss of the internals indexing fixture (1IF) level indication system (bubbler) will result in an erroneous level indication which will be noted when compared with a redundant level indication system. Since this system has no control features, no adverse system conditions will result.

> Page 20 Rev. 2 0057Y

Loss of instrument air will take the individual components to their fail safe position. Flow mismatches induced by loss of air will result im automatic trips. Loss of air to the lIF level monitoring system will initiate a low air supply pressure alarm.

On the loss of instrument air, level control in the ion exchangers would be lost; however, the ion exchanger inlet isolation valve (V025 & V260) would fail to the closed position. Also, the level control valves, V029 and V266 (LV45 & LV72), will fail to the closed position to prevent siphoning of the ion exchanger.

# 4.3 Filter Media Rupture

A failure of the filter media in the canister could potentially release fuel fines to the ion exchange portion of the system. A post filter is located downstream of both filter trains in the line to the ion exchangers. This filter will trap any fuel fines which would be transported past the filter canisters in the event of filter failure. The post filter is designed to be critically safe and is sized so that a small accumulation of debris will increase the differential pressure to the alarm setpoint. Also, the nephelometers in the return line would alert the operator to a possible media rupture since the turbidity would increase rapidly.

The recovery procedure is to isolate the filter trains and find the ruptured filter by observing the differential pressure versus flow for each individual canister. Lower differential pressure for a given flow will indicate that this filter is ruptured. That canister or canisters and the post filter rartridge would be replaced and the system restarted.

#### 4.4 Line and Hose Break

The consequences of any line and hose break is a loss of reactor vessel inventory. The system has been designed to mitigate the consequences of such an incident to the extent possible.

To help prevent a hose rupture, all process hoses are armoured. In case of a hose rupture or line rupture, downstream of the reactor vessel pumps, P-2A & 2B, the system is equipped to trip these pumps on the IIF low level and alarm to the control panel. This event could deliver approximately 5CO to 1000 gallons of reactor vessel water to the area of the break. The potential areas affected would be the Reactor Building and the Fuel Handling Building, each of which has sumps or drains to the Aux. Bldg. sumps to contain the spill.

> Page 21 Rev. 2 0057Y

If a suction hose to the well pumps or a return hose to the reactor vessel should rupture, a siphoning of reactor vessel water would take place. The two 4 inch suction connections provided in the Westinghouse work platform are provided with two 3/4 inch holes drilled 18 inches below the water level which will act as a siphon breaker. The three 2 inch return lines are equipped with spargera, which are simply holes drilled into the pipes. The first holes are drilled 18 inches below the water level which will act as a siphon breaker. The sample return line is terminated 18 inches below the water level. Therefore, a maximum of approximately 3000 gallons of reactor vessel water would spill into the fuel transfer canal following a hose rupture. Approximately half of this water would be contained in the New Fuel Pit.

The recovery from these events would be accomplished by isolating the ruptured section and replacing the ruptured hose/pipe.

## 5.0 SYSTEM MAINTENANCE

The maintenance procedures are the recommended practices and intervals as described by the equipment vendors. This section will be expanded as equipment selection is finalized and vendor information becomes available.

#### 6.0 TESTING

11-1-1

#### 6.1 Hydrostatic Testing

Piping and hose will be hydrostatically pressure tested. Testing of hose will be done after couplings have been attached. Pipe will be tested outside the buildings.

#### 6.2 Leak Testing

All accessible connections will be initial service leak tested after the piping is assembled.

#### 6.3 Instrument Testing

All instruments will be calibrated by vendors. Complete electric/pneumatic loop verification will be done during start-up.

Page 22 Rev. 2 0057Y

#### 7.0 HUMAN FACTORS

Filter canister hoses are coded for quick identification of inlet versus outlet.

Extensive use of hoses is made, especially in the Reactor Building, Allowing quick installation and use of existing radiation shielding. Hoses which are expected to be frequently disconnected are equipped with quick disconnect couplings for ease of removal and replacement.

The following human factors guidelines have been incorporated into the design of the DWCS control panel:

- The panel includes all controls and displays required for normal operation.
- Displays provide immediate feedback that the system has responded appropriately to an operator's action.
- o Controls and displays are laid-out for a left to right flow path.
- Mimic lines are used to clarify flow paths.
- o Control devices are mounted 3 to 6 feet above the floor.
- o Each control device has a name plate.
- o Light bulbs are replaceable from the front of the panel.
- Recorders are grouped on the right side of the parel away from the flow path.
- Adjustments to recorders and controllers can be performed from the front of the panel.